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LA-UR-89-1333

Received

MAY 09 1989

LA-UR--89-1333

DE89 011173

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**TITLE: MEASURED NEUTRON BEAM LINE SHIELDING
EFFECTIVENESS OF SEVERAL IRON/POLYETHYLENE
CONFIGURATIONS**

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**SUBMITTED TO: Contributed Conference Paper to be published in the
proceedings of ICANS-X, October 3-7, 1988, Los Alamos, NM**

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Measured neutron beam line shielding effectiveness of several iron/polyethylene configurations

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ABSTRACT: Neutron and gamma-ray leakage measurements were taken at various stages of shield construction for neutron flight path 5 (the Lash-up flight path) at LANSCE, to compare the relative effectiveness of several configurations. Dose equivalent rates were determined for three categories: "low-energy neutrons", below 20 MeV; "high-energy neutrons", above 20 MeV; and gamma rays, as measured by hand-held survey instruments. The low-energy neutrons were measured by activation of an indium foil in a paraffin-filled cadmium canister, sized to be generally insensitive above 20 MeV. High-energy neutrons were measured by (n,2n) production of Carbon 11 in a plastic scintillator with a 20-MeV threshold. Thermal neutrons were not measured at the shield-leakage test points. Room-scattered neutrons were observed by Albatross IV detector readings, which were taken beside the shield as a measure of variation of room background as the shield configuration changed.

A sketch of the final shield cross section is shown in Figure 1. For these tests, the walls were completed as shown, and the top was added in stages. Measurements were taken directly above the beam pipe at two and four feet from the LANSCE bulk shield. The four-foot data is more representative of shielding a beam pipe with neutron collimators, as opposed to data taken nearer the inefficient interface between the flight path shield and the bulk shield. We think that a strong component of high-energy neutrons impinges on the beam path collimators and nearby shielding because of lack of adequate high-energy collimation within the bulk shield. Thus, the source is strengthened by spallation and evaporation spectra within the flight-path shield.

The data tabulated in Table 1 are generally self explanatory. "P" refers to commercial high-density polyethylene, typically of specific gravity 0.94. "BP" refers to five-wt.-percent borated polyethylene of generic commercial varieties. One important indication of these data is the utility of efficient lamination and relative thicknesses of shield materials.

Although the data represent a small and perhaps specialized sample, one interesting finding is that the "plain" polyethylene reduces the neutron and total doses (in spite of the 2.2 MeV gammas) more than the borated polyethylene. This effect is not unexpected for shields of this thickness, where a strong high-energy component occurs in the source; addition of the boron compounds displaces ~25% of the hydrogen in the polyethylene. The resulting effect is strongly source-dependent and may also be quite geometry-dependent, considering neutron buildup and reflection from opposite walls within the shielded cavity. Gary Russell has performed leakage calculations for a large number of shield configurations, reported elsewhere, which show the interplay of these effects.

ACKNOWLEDGEMENTS

This work was done with the scheduling support and encouragement of Dick Woods, A. Jerry Miller, and Roger Pynn, and with financial support of the U. S. Department of Energy, Office of Basic Energy Sciences. We acknowledge useful discussions with Gary Russell and typing support of Teri Cordova.

| Table I | | | | | | | | | |
|---|------|-----|-----|-------|------|-----|-----|-------|-------------|
| FLIGHT PATH FIVE SHIELD TEST RESULTS, OCTOBER 25-31, 1987 | | | | | | | | | |
| mrem/hr | | | | | | | | | |
| SHIELD CONFIGURATION INCHES | 2 FT | | | | 4 FT | | | | SIDE LEN |
| | LEN | HEN | G | TOTAL | LEN | HEN | G | TOTAL | |
| 6Fe | 1540 | 32 | 160 | 1730 | 1000 | 36 | 104 | 1140 | 23 |
| 6Fe,4BP | 355 | 36 | 40 | 425 | 116 | 27 | 14 | 157 | 6.1 |
| 12Fe | 1340 | 14 | 40 | 1400 | 787 | 10 | 25 | 822 | 20 |
| 12Fe,12BP | 10 | 8 | 4 | 22 | 8 | 4 | 2 | 14 | 6.1 |
| 12Fe,2P,6Fe,4P | 2.6 | 3.6 | 2.1 | 8.3 | .9 | 1.2 | 1.1 | 3.2 | 0 |
| 12Fe,2P,6Fe | 53 | 5.4 | 3.6 | 61 | 28 | 2 | 1.6 | 32 | 0.4 |
| 12Fe,2P,6Fe,4BP | 4.7 | 3.4 | 1.6 | 9.7 | 3.2 | 1.8 | 0.6 | 5.6 | 1.4 |
| Shutters Closed | 0.6 | 6 | 0.4 | 7 | 0.6 | 6.4 | 0.4 | 7.4 | 3 |
| Background | | | | | | | | | |

HEN = High-energy neutrons, > 20 MeV, measured by carbon scintillator activation, using flux-to-dose rate conversion factor (5 n hr/mrem cm² s)

LEN = Low-energy neutrons, < 20 MeV, measured by moderated indium foil activation, using flux-to-dose rate conversion factor (7 n hr/mrem cm² s)

Thermal neutrons (< 3% of dose) not measured

Background (FP 4 and FP 5 shutters closed) subtracted from data

